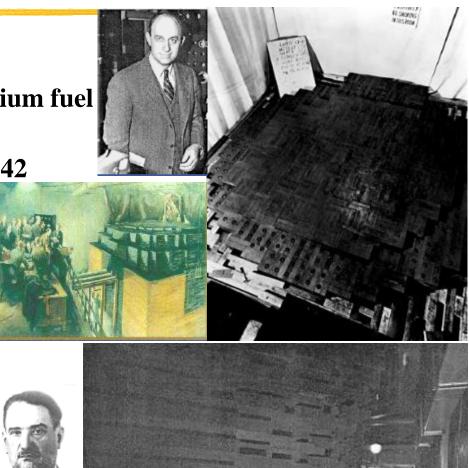
Kjernekraft i Norge?

Dieter Roehrich UiB

Shall Norway invest in nuclear energy and build nuclear reactors?

History First reactors

- graphite moderated, natural uranium fuel
- 1. E. Fermi, Chicago Pile, USA, Dec. 1942



2. I. Kurchatov, Russia, Dec. 1946

Controlled chain reaction of neutron induced fission processes of uranium or plutonium nuclei

- Fuel cycle
 - ^{235}U is the only natural fissile material, natural uranium contains 99.3% ^{238}U and 0.7% ^{235}U
 - the other isotopes (²³³U from ²³²Th), (²³⁹Pu from ²³⁸U) have to be produced in reactors ("breeding").

Reactor technology

- Reactor design: **thermal reactors** (slow neutrons) or **fast reactors** (fast neutrons)
- Coolant

The energy released in the fission process is converted into heat which has to be transferred away from the reactor core by a coolant. Typical coolants for thermal reactors are water or helium gas; fast reactors used liquid metals (sodium or lead)

Moderator (only thermal reactors)
 In case of a thermal reactor, the fast neutrons have to be slowed down by a moderator.
 Moderator and coolant can be identical, but don't have to be. Typical moderators are water (normal or light water), heavy water (made with deuterium) and graphite.

Nuclear energy is a complex technology with many risks

Four key problems:

- **1.** Operational accidents
- 2. Shortage of ²³⁵U fuel / Breeding of ²³³U from ²³²Th and ²³⁹Pu from ²³⁸U?
- 3. Waste management
- 4. **Proliferation nuclear weapons**

1. Operational accidents

- Criticality accident (Chernobyl)
 loss of control of reactivity → prompt criticality
- Loss of coolant (Fukushima)
 Energy release after shutdown (normal operation: 2700 MW_{th})

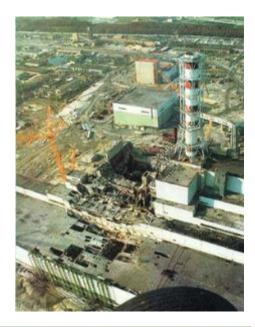
After 1 minute: 150 MW

After 1 hour: 45 MW

After 1 day: 15 MW

Weeks/months: ≈15 MW

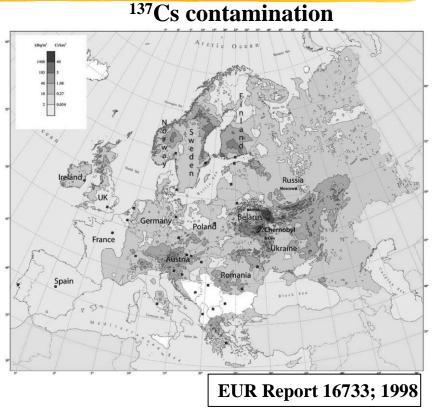
→ The heat must be removed from the reactor core after shutdown of the chain reaction.
 Without cooling, the fuel rods overheat, react with water/steam creating hydrogen and finally melt.





Criticality accident (Chernobyl) - consequences

- Release of 1-2% of the radioactive inventory into the atmosphere
- Radioactive contamination of central, eastern and northern Europe
- Collective dose in the northern hemisphere:
 ≈ 600.000 person-Sv
 IAEA-SM-339/185; 1995
- Estimates of excess fatal cancer cases based on the Linear Non-Threshold (LNT) model:
 - 1. Risk factor per Sv: $1.5\% \rightarrow 17.850$ cases
 - 2. Risk factor per Sv: 5-10% \rightarrow 30.000-60.000 cases

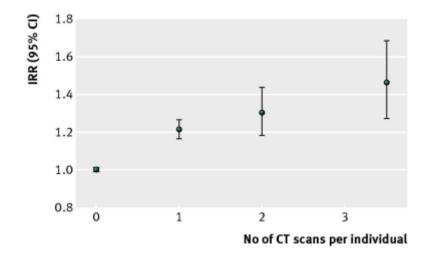


Int. J. Cancer: 119, 1224-1234; 2006

Medicine, Conflict and Survival, 23:1; 2007

Establishing a dose-response relationship

- Linear non-threshold dose-response curve: epidemiologic evidence
 - Radiation exposure from CT scans in childhood
 - Incidence rate ratios (IRR) for all types of cancers in exposed versus unexposed individuals vs the number of CT scans (≈5.7 mSv per scan)



BMJ 2013;346:f2360 doi: 10.1136/bmj.f2360

2. Shortage of ²³⁵U fuel - reported uranium reserves last until about 2040 Solution to the fuel crisis: breeding of ²³³U from ²³²Th and ²³⁹Pu from ²³⁸U

- Thermal Thorium breeder: conversion factor of only 80%, i.e. no breeding
- Fast Plutonium breeder: breeding factors higher than one have been achieved

Compact reactor core – just fissile and fertile fuel rods

 \rightarrow high neutron flux \rightarrow material damage

- \rightarrow highly enriched fuel (²³⁹Pu, ²³³U)
 - \rightarrow narrow range of allowed reacticity \rightarrow criticality accidents

 \rightarrow proliferation

 \rightarrow High energy density – liquid metal coolant (sodium, lead)

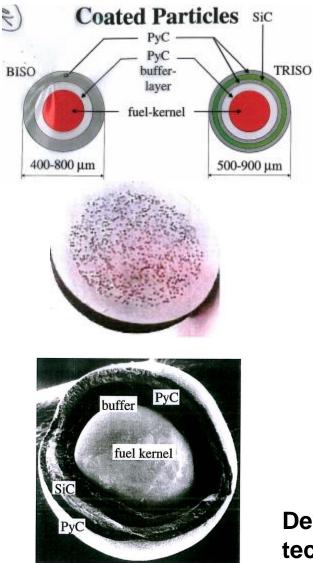
 \rightarrow unproven technology (on large scale) \rightarrow accidents

 \rightarrow Requires reprocessing of spent fuel - closed fuel cycle

 \rightarrow hot chemistry \rightarrow (criticality) accidents

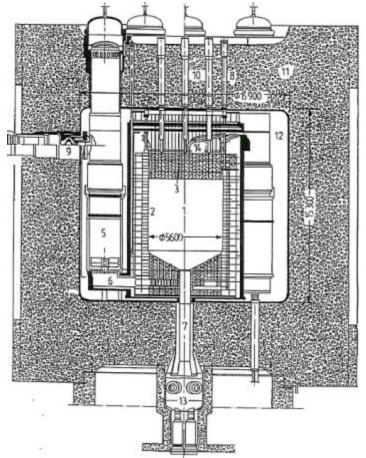
 \rightarrow proliferation

Thorium high temperature reactor



Gas-cooled (He) graphite-moderated reactor

- Fuel:
 675,000 spherical fuel elements
- Fuel element: 30,000 coated particles
- Fuel elements are continuously loaded during operation
- They are recycled several times (about 6) to gain the final burn-up



Development costs for the fuel cycle and the reactor technology today: $\sim 50 - 100$ billion NOK

Reactor operation diagram: safety margins

235**T** J stable reactor stable reactor 233TT stable reactor ²³⁹Pu period period T [sec] T [sec] period T [sec] with delayed neutrons with delayed neutrons with delayed neutrons prompt neutrons only prompt neutrons only prompt neutrons only 10^{3} 10^{3} 10^{3} prompt prompt prompt critical critical critical 10^{2} 10^{2} 10^{2} region region region 10 10 10 $= 10^{-10}$ $\tau = 10^{\circ}$ $\tau = 10^{-3}$ $\tau = 10^{-4}$ $\tau = 10^{-1}$ $\tau = 10$ 10^{-} 10^{-1} 10 10^{-} 10^{-2} 10- 10^{-} 10^{-3} 10-10-4 10-3 10-3 10 10 10 10 reactivity reactivity reactivity

Thermal reactors

Fast reactor

3. Waste management

Fuel inventory of a typical power reactor (LWR, 1 GW_{el}): 100 tons Spent fuel discharge per year: 30 tons containing about 2% unused ²³⁵U, plutonium isotopes of breeding process from ²³⁸U and fission products: 600 kg ²³⁵U

285 kg Pu (70% ²³⁹Pu) 450 kg fission products

- **Fuel cycles**
- Open fuel cycle (once-through)

uranium ore -> enrichment -> reactor -> waste storage (current reactor technology relies on enriched ²³⁵U fuel)

- Closed fuel cycle

²³⁸U - ²³⁹Pu or ²³²Th - ²³³U cycles, require reprocessing of spent fuel

Reprocessing of spent fuel - ²³⁹Pu/²³³U extraction

Medium/large-scale chemical plant

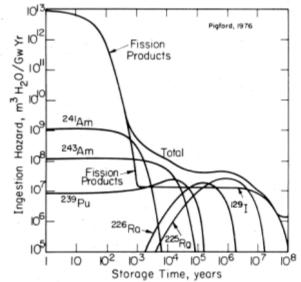
- PUREX: various "hot" chemical processes
 - chopping up spent fuel
 - dissolving the fuel in acid
 - solvent-extracting and ion-exchanging processes
 - converting plutonium to metallic form
- THOREX: similar process for the extraction of ²³³U

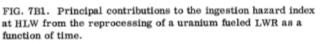
→ accident prone, high maintenance hot chemistry plant (e.g. Sellafield)

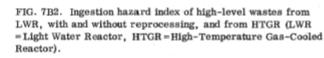


Storage of nuclear waste

- Waste from LWR (U fuel), with and without reprocessing, and from Thorium reactors (U-Th) Pigford, 1976
 - some differences in toxicity after 200 years
 - waste has to be kept away from biosphere and/or safeguarded for about 10⁷ years







U Fueled LWR;

0.5 % U and Pu in Wastes

Storage Time, years

Dischärge U Fuel

LWR With Self-generated Pu Recycle, 0.5 % U and

106 107

HTGR

108

Pu in Wastes

from LWR

510

10

10

U-Th

Fueled HTGR

m³ H₂0/Gw)

Ingestion Hazard,

in

10

- Storage in a geologic repository
 - no consensus on what is a safe geologic formation

4. Proliferation

Any civilian nuclear installation (enrichment plant, reactor, reprocessing plant) can give access to weapons-grade nuclear material

- Enrichment plant no difference in operation for 3% or >80% (weapons-grade) enrichment of ²³⁵U
- Reactor operation short burn-up of fuel gives high yields of weapons-grade isotopes (²³⁹Pu and ²³³U)
- Reprocessing plant PUREX and THOREX process - extraction of almost pure ²³⁹Pu and ²³³U





Nuclear energy is a complex technology with many risks

- **1.** Some experience with thermal reactors and ²³⁵U fuel
- 2. Very limited experience with breeding of ²³³U from ²³²Th and operating reactors with ²³³U

one large-scale prototype in Germany (1960-86) \rightarrow conversion factor of only 80%, i.e. no breeding

- 3. Some, mostly negative, experience with fast reactors for breeding of ²³⁹Pu from ²³⁸U
- 4. Closed fuel cycle, i.e. chemical reprocessing of spent fuel, is messy
- 5. Waste storage not solved
- 6. Norway no longer has expertise in nuclear energy and there is no nuclear industry
- → Instead of spending money on nuclear energy, Norway should invest in lossless energy transmission and energy storage technologies

The End